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Partitioning of Fission Products and Waste Salt Minimization during Pyroprocess

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KAERI

Backgrounds

Next generation nuclear fuel cycles

• Reduction of environmental hazard by selectively recovering a long-lived nuclide and transmuting it

Minimization of the waste volume to be eventually disposed of

Basic strategy for partitioning

- Classify and optimize waste streams arising when treating spent fuel
- Minimize waste volumes resulting from each waste stream

Little information on a strategy for waste minimization in pyroprocess

Strategy for optimization of waste streams and waste minimization

- Pyro-partitioning of fission products being performed in KAERI
- Strategy requested for optimizing waste streams and minimizing waste amounts
- Reduction of HLW generating from each waste stream
 - -Converting HLW to LLW through an increase of DF or SF
- Candidate wasteforms for consolidation of waste salts from a pyroprocess

A Strategy for Efficient Management of Spent Fuel in Korea



FP: Rare earth, Noble metal, Volatile / Semi-volatile Fission Products KAERI's waste management strategy: directed to minimize HLW amounts

A Flow Diagram of Pyroprocess being Developed by KAERI

Optimizing waste streams and evaluating a strategy for minimization of wastes



Fission Products Release with Voloxidation Conditions

Nuclides	Standard voloxidation (500°C)	Advanced voloxidation* (1200°C)	Remarks	Flow sheet for capturing fission gases			
Kr / Xe H	<30% 100%	100% 100%		Advanced VoloxidizerCs,Rb,Cd Trapping UnitTc, Ru,C-14 Trapping UnitHT Conversion UnitI1200 cCFitCs,kessel fitCropping UnitCropping UnitTrapping Unit			
I	<10%	100%	l-129: long- lived nuclide	~1200 °CFly ash FilterCa-based filterCuOAg-Xvacuum1,000 °C600 °C400 °C150 °C			
Тс	<1%	92%	Tc-99: long- lived nuclide				
С	<10%	100%	C-14: long- lived nuclide	HTO Trapping Full HEPA Trapping			
Cs	<1%	98%	Highly radioactive, high decay heat nuclide	Unit Filter Unit Molecular RT Solid Adsorbent Sieve, RT RT -80 °C			
Ru	<1%	98%	Noble metal				
Мо	<1%	62%	Noble metal	Challenges			
Rh	<1%	83%	Noble metal	\sim DE > 10.4 to minimize the impact to the environment			
Rb	<1%	96%		Further development of trapping technologies and condition			

- * INL hot experimental data in I-NERI program of INL-KAERI Advantages
 - Minimize influence of fission products on the down-stream process conditions
 - ✓ Recover and store fission products separately

- Optimization of waste forms for consolidation of several absorbents
- Minimization of waste amounts issued from capturing fission gases

Treatment of Hull

Objective:

- > Experimentally recover higher than 99% fissile material during air-voloxidation process
- Look for a promising way enabling a conversion of hull to LLW
- Strip residual fissile materials from the contaminated hull
- ➤ Classify the hull as LLW → Challenge !!



1) Steven M. Frank, et al., "Immobilization of Technetium and other Fission Products from Processed Spent Nuclear Fuel into a Metallic Waste Form", 2008 IPRC, Aug 24-27, 2008 Jeju Island, Korea 6

Waste Salt Treatment Technologies

Developing technologies to recycle waste salts to process units, not by releasing to repository
 -Salt cooling technologies: Czochlarski, Zone freezing, Layer crystallization
 -Precipitation technology: oxidation-precipitation using air



Recovery of Purified LiCI Salt from a Waste Salt



Czochlarski



Layer Crystallization



Zone Freezing



- To concentrate Cs and Sr to eutectic point by cooling method
- To theoretically recover 99wt% of LiCl from a waste salt
- To evaluate and select which technology is more preferable and effective for scale-up and practical use.

Zone Freezing Technology for Recovery of LiCI Salt



Experimentally recycle 90% of LiCl (contaminated with

a small amount of

impurity)

Fabricated to a final wasteform (contaminated with 90% of initial impurity amount)

LiCI-KCI Waste Salt Recycle

Possible to recycle all most of eutectic salts to electrorefiner and to minimize waste salt to be disposed of

REE are precipitated as oxide or oxychloride forms



Wasteforms



A Flowsheet for Wasteforms Fabrication



Chemical route and Qualification of Waste forms

Chemical route for the immobilization of each waste For LiCl waste

Salt waste + 2 SAP \rightarrow Li_xAl_xSi_{1-x}O_{2-x} + Li₃PO₄ + (Li, Cs)-aluminosilicate + Cs₂AlP₃O₁₀ +

 $\frac{Sr_5(PO_4)_3CI(apatite) + CePO_4(monazite) + amorphous phase (M_2O-AI_2O_3 - P_2O_5) + CI_2 (Vaporized as gas)}{P_2O_5}$

For REE oxide waste $RE(An)_2O_3 + 2 NH_4H_2PO_4 = 2 RE(An)PO_4 + 2 NH_3 + 3 H_2O$

Waste loading factor of waste forms

LiCl waste salt : SAP material : Glass frit = 1 : 2 :1 [waste loading : 25 wt%] for SAP RE oxide : Chemical additive($NH_4H_2PO_4$) : Glass frit = 1 : 0.52 : 3.48 [waste loading: 20wt%] for Monazite

Chemical durability of waste forms

PCT-7days,

(unit : g/m²)

	SAP	Monazite	Conventional	Sodalite	
	for LiCI waste	for REE oxides	BSG	for LiCI-KCI	
Cs	~ 10 ⁻²	-	~ 10 ⁻¹	~10 ⁻²	
Sr	~ 10 ⁻²	-	~ 10 ⁻²	~10 ⁻²	
RE	-	< 10 ⁻³	< 10 ⁻³	-	

Partitioning-Waste Streams

	Waste stream	Partitioning & Recovery	Potential Wasteform	Classification
	Volatile / Semi- volatile species	Selective partitioning of FP's using substrates	Not determined	LLW
Spent fuel	Hull Metal sludgy (NM)	Removal of residual actinides from metallic products	Alloy form with Zr and Fe (may include Tc and other noble metals)	HLW →LLW
Pyroprocess	Cs / Sr + LiCl salt	Recycling of LiCl to Electroreducer	Ceramic waste (SAP)	HLW(?)
	Lanthanides + LiCI-KCI (with TRU)	Oxide precipitation & salt recycled to ER	Ceramic waste (Monazite)	HLW
	U	Removal of actinides (contaminated with > 100 nCi-TRU/g) to store as LLW	Temporally stored as Metallic form	LLW
	TRU	Recovery yield: > 99.9 %	Transmuted at ABR	

Conclusion

Classify into 4 waste streams

-Volatile and semi-volatile species waste stream

-Metallic form waste stream

-Uranium metal waste stream

-Salt waste stream-ceramic composite wasteform

*** KAERI'** strategy for waste minimization is focused on

-Reduction of HLW by converting to LLW

Removal of actinide residual from hull
Removal of actinides from an electrorefinned uranium metal product
Increase of decontamination factor of actinide from waste LiCl salt
Reduction of LLW: waste arising from voloxidation
Simplification of wasteforms applicable to several absorbents
Reduction of waste amounts

Comparison of wasteform volume with and without a salt recycle -Possible to reduce HLW amounts by 30 times -Predictable to reduce HLW by 80 times if converting LiCl waste to LLW

Eng.-scale demonstration for waste salt recovery and wasteform fabrication -Establishment of Eng.-scale facility(10 t-HM/yr) for an inactive test by 2011 at KAERI



Thank for your attention !!

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