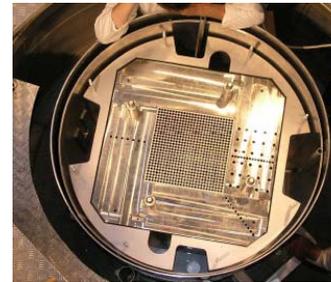
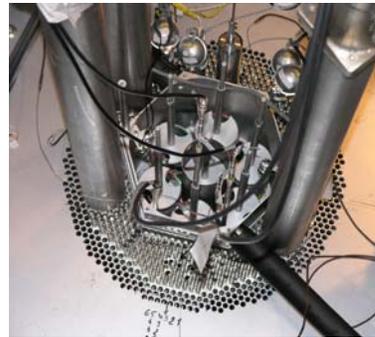


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# NUCLEAR DATA PRODUCTION, CALCULATION AND MEASUREMENT: A GLOBAL OVERVIEW OF THE GAMMA HEATING ISSUE



WONDER-2012 | A.C. Colombier, H. Amharrak, D. Fourmentel, S. Ravaux, D. Régnier, O. Gueton, J.P. Hudelot, M. Lemaire  
CEA, DEN, DER, Cadarache

[www.cea.fr](http://www.cea.fr)

25-28 september 2012

1. Context

2. Nuclear data production

3. Simulation Codes

4. Integral experiments and instrumentation

5. Conclusion

# 1. CONTEXT

## Nuclear heating

### Energy deposition :

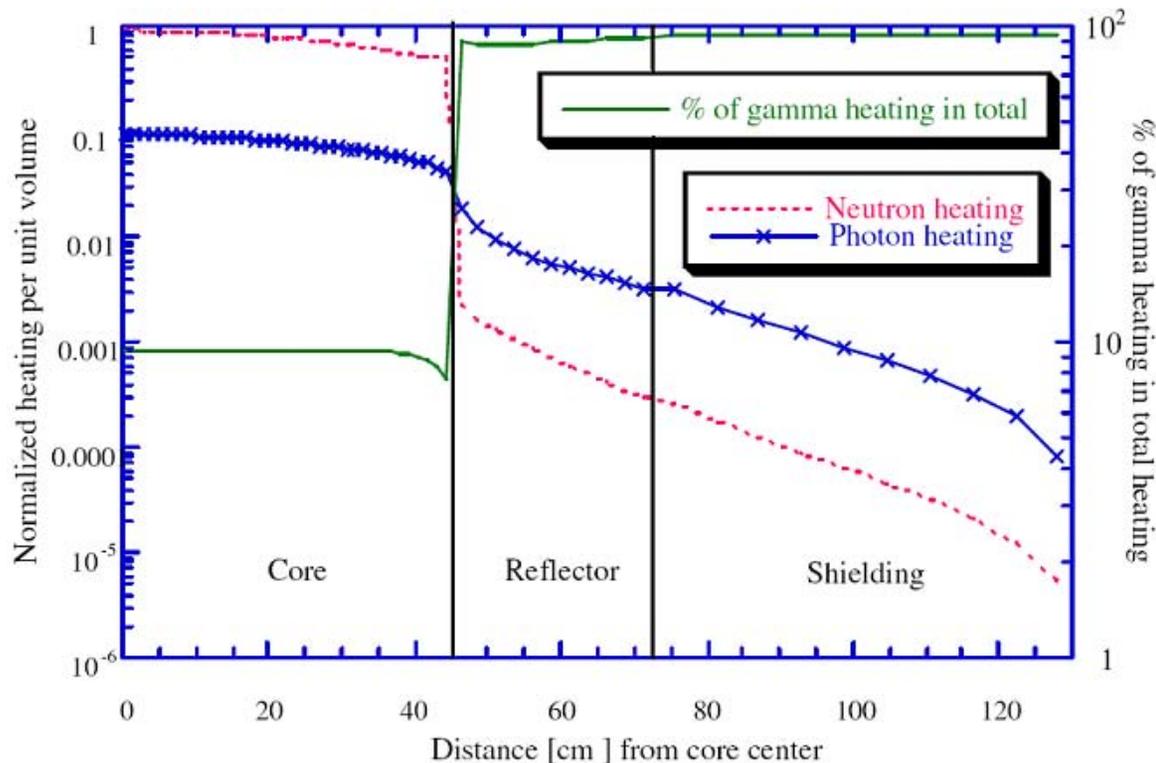
charged particles ( $\alpha$ , proton,  $e^\pm$ ), ions from n &  $\gamma$  interactions

### Photon production :

- . neutron interactions (fission, radiative capture, inelastic scattering)
- . fission products decay

### Photon interactions :

- . photoelectric effect
- . Compton scattering
- . pair production



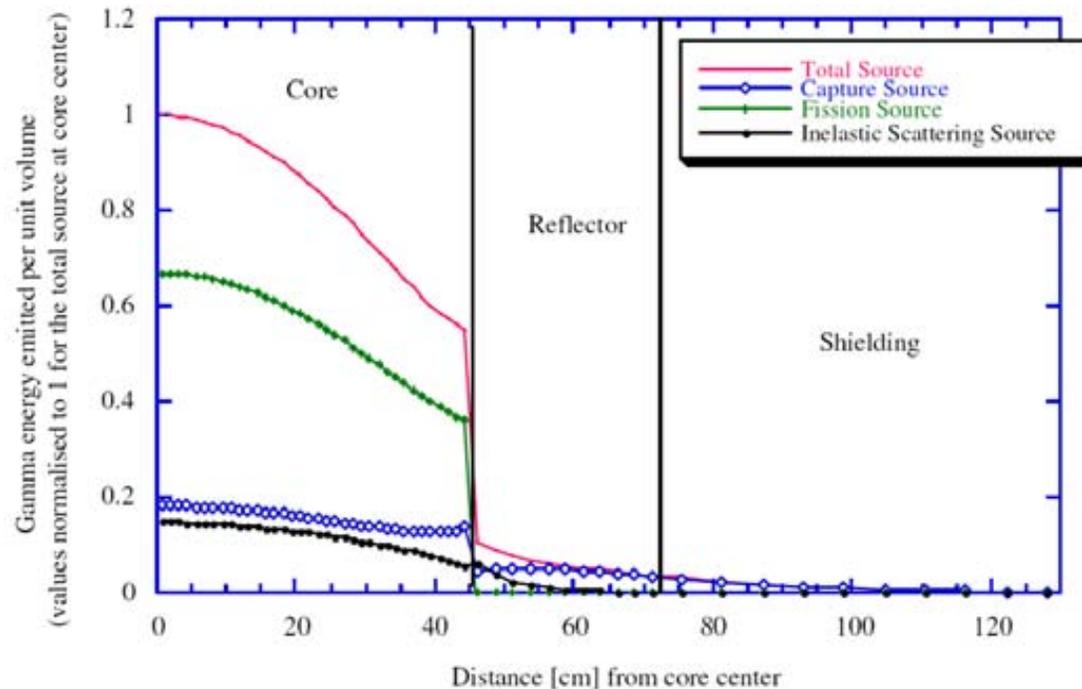
In the core : > 90% -> neutron (local deposit)

In the reflector : > 90% -> photon

# CONTEXT – Nuclear $\gamma$ heating in power reactor

## Relative contribution in a fast reactor

- Fission : 70% (~ 30% delayed gamma)
  - Radiative capture : about 20%
  - Inelastic scattering : the rest
- } Depending on the reactor material balance



## New concept New design

### PWR – GEN III+ :

stainless steel heavy reflector  
Burnable poison ( $\text{UO}_2\text{-Gd}_2\text{O}_3$  pins for PWR...)

### GEN IV – Fast Reactor :

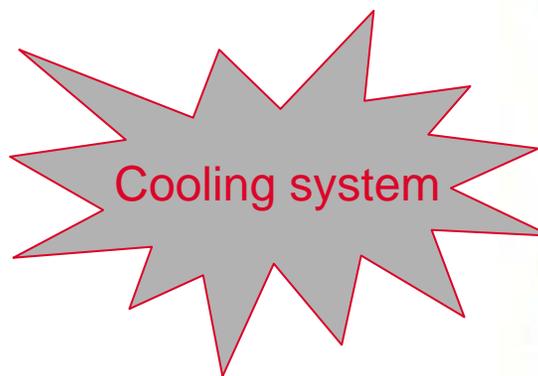
steel control rod followers and diluent

### MTR – JHR :

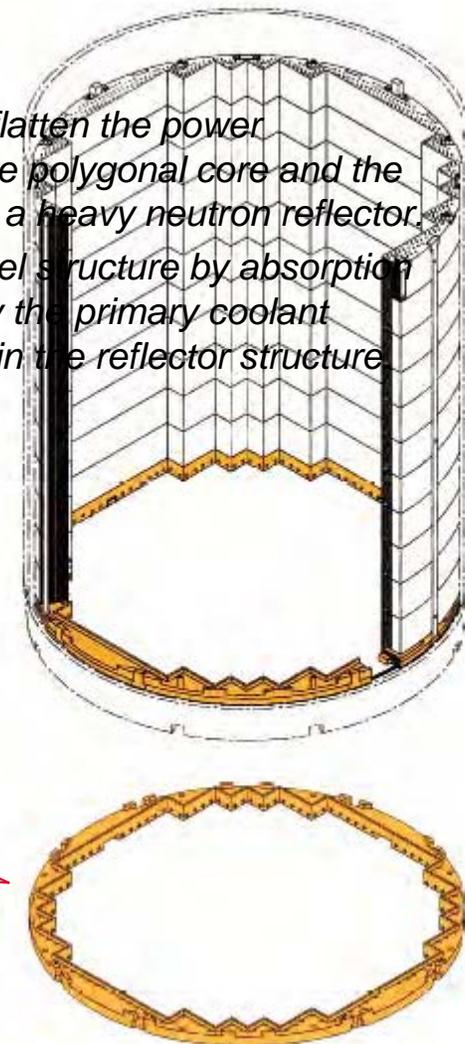
Hf control rods  
Aluminium structure  
Beryllium reflector  
Cd shielding  
design of experimental devices

*Safety* *lifetime*

*Performance*



*To reduce neutron leakages and flatten the power distribution, the space between the polygonal core and the cylindrical core barrel is filled with a heavy neutron reflector. The heat generated inside the steel structure by absorption of gamma radiation is removed by the primary coolant through holes and gaps provided in the reflector structure.*



- Nuclear data evaluations
- Experimental measurement uncertainties
- ➔ better description & simulation



**5 PhD works at**



**1 collaboration : McMaster University (Canada)**

**2009 : Gamma heating Working Group**

## **2. NUCLEAR DATA PRODUCTION**

## Nuclear data library : JEFF3.1.1

Missing data for all n interactions<sup>1</sup> :

Gd, Cd, Ag, In, ...

Incoherent  $Q_\gamma$  for  $(n,\gamma)$ <sup>1</sup> :

54,56,57,58 Fe, <sup>62</sup>Ni

Old fashioned or missing data for  
prompt gamma fission production<sup>2</sup> :

<sup>235</sup>U, <sup>239</sup>, <sup>241</sup>Pu

## New evaluations

$(n,\gamma)$   $(n,n')$ <sup>1</sup> : gamma emission spectrum

. Discrete level : EGAF/ENSDF libraries

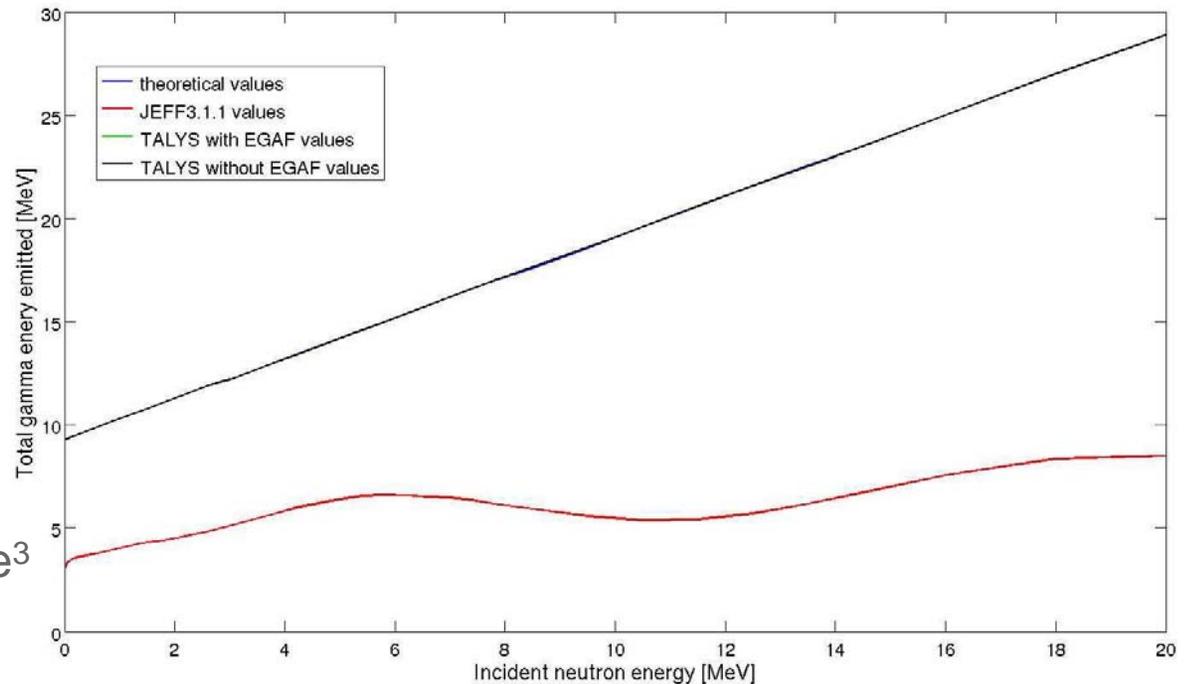
. Continuum ( $A > 70$ ) : nuclear reaction  
code TALYS (RIPL3.0 param. library)

Monte Carlo simulation code : FIFRELIN<sup>2</sup>

<sup>1</sup> S. Ravaux, D. Bernard, A. Santamarina, *New evaluation of photon production for JEFF3*, Conf. PHYTRA2, (2011)

<sup>2</sup> D. Regnier, O. Litaize, O. Serot, *Monte Carlo simulation of prompt fission gamma emission*, Phys. Proc. 31, 59-65, (2012)

Integrated gamma production spectra :  $(n,\gamma) ^{54}\text{Fe}$



Discrepancy > 50% !

➤ 16 stable isotopes available<sup>3</sup>

<sup>3</sup>S. Ravaux, D. Bernard, NEA Data bank, JEFDOC-1404 (2011)

## FIFRELIN: Monte Carlo simulation of fission fragments evaporation

+ new module :  
de-excitation gamma cascade simulation

- density level distribution model
- strength function

+ preliminary tests :

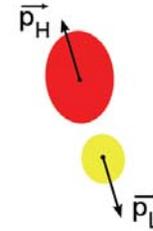
- $^{55}\text{Mn}$  ( $n,\gamma$ )
- $^{144}\text{Ba}$  fission fragment de-excitation
- $^{252}\text{Cf}$  spontaneous fission

Fissioning nucleus



T:

Figure 1:  
Compound nucleus



T:

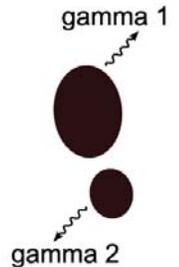
Figure 2: Fully  
accelerated  
fragments

neutron 2



T:

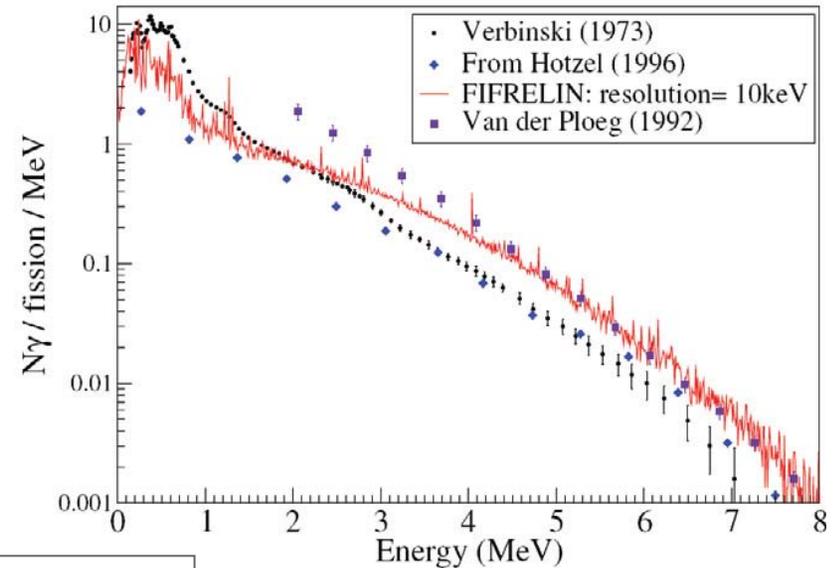
Figure 3: Prompt  
neutron emission



T:

Figure 4: Prompt  
gamma emission

(T= nuclear temperature)

$^{252}\text{Cf}$  prompt gamma spectrum

+ preliminary results :

➤ Discrepancy source :

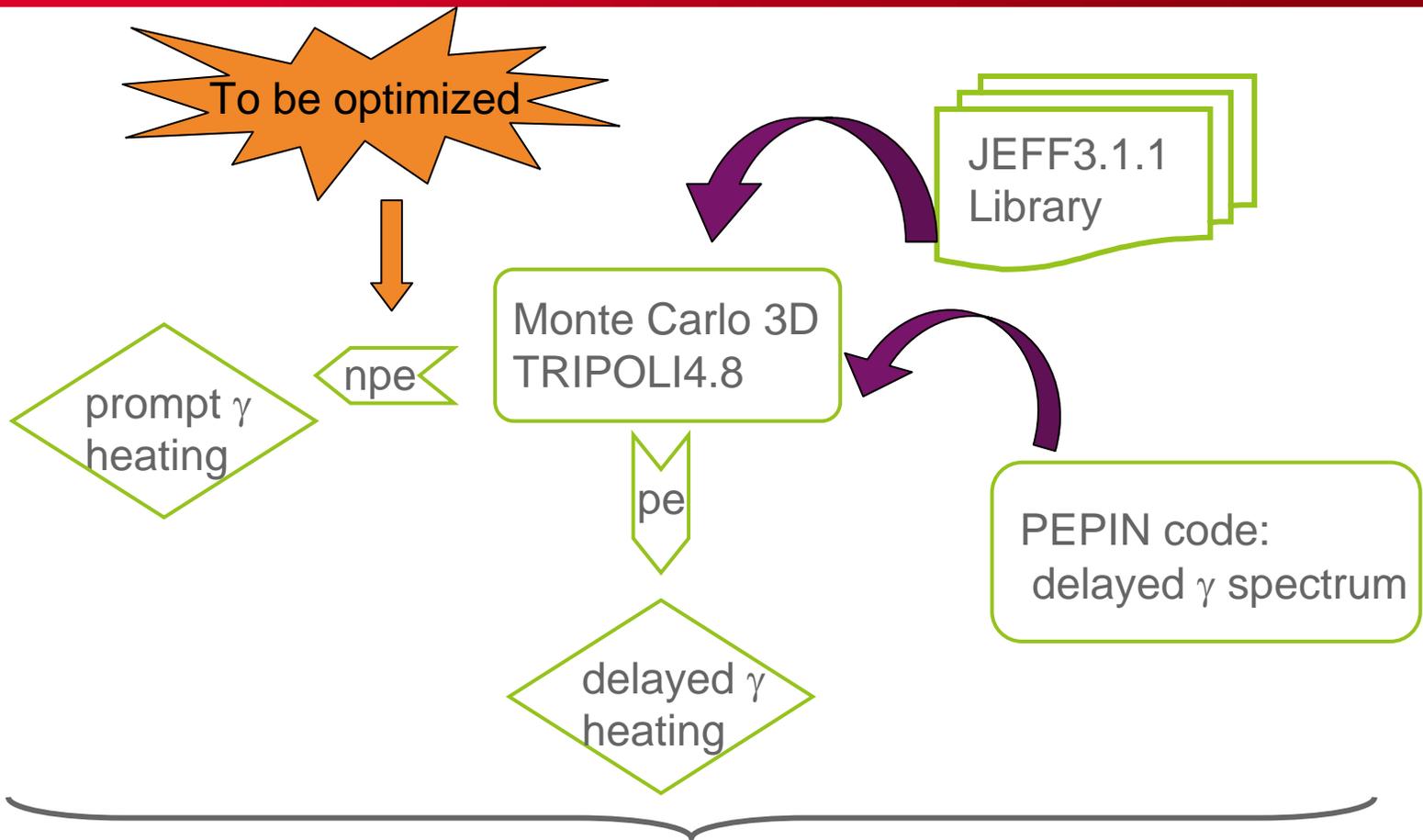
density level distribution model > strength function

➤ Qualitative agreement : good

➤ Quantitative analysis: systematic overestimation

## **3. SIMULATION CODE**

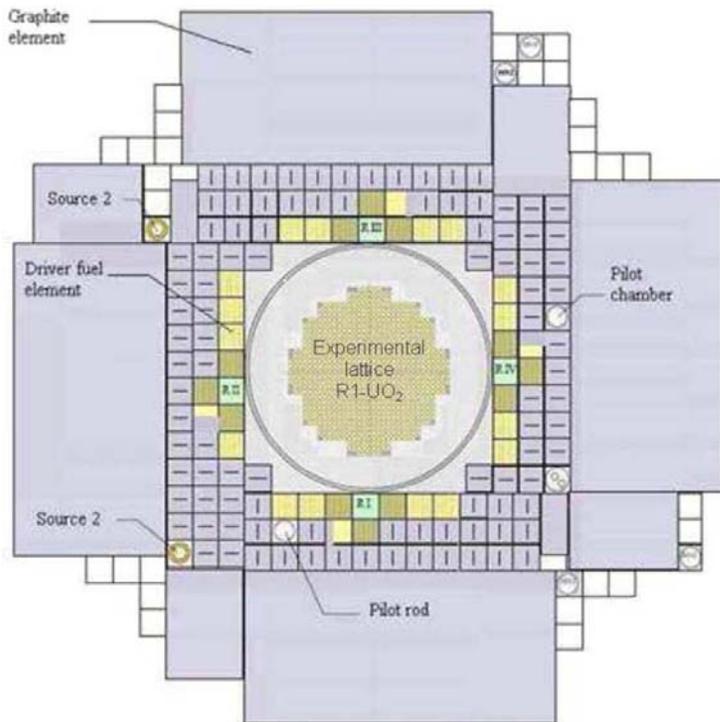
# SIMULATION CODE



np APOLLO2-MOC QUALIFICATION

## **4. INTEGRAL EXPERIMENTS AND INSTRUMENTATION**

MINERVE facility : R1-UO<sub>2</sub>  
PWR standard spectrum

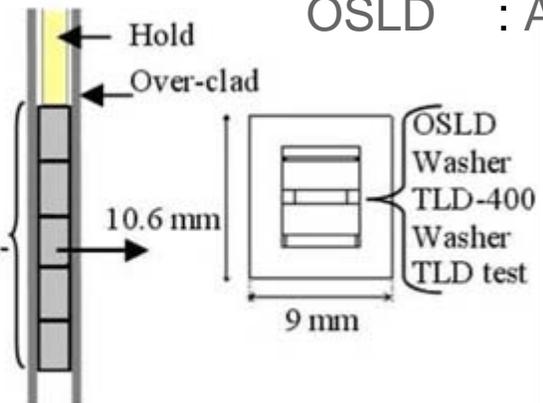


γ heating measurement in zero power reactor

- 
- Thermo-Luminescent Detector TLD
- Optically Stimulated Detector OSLD

TLD-400 : CaF<sub>2</sub>:Mn  
OSLD : Al<sub>2</sub>O<sub>3</sub>:C

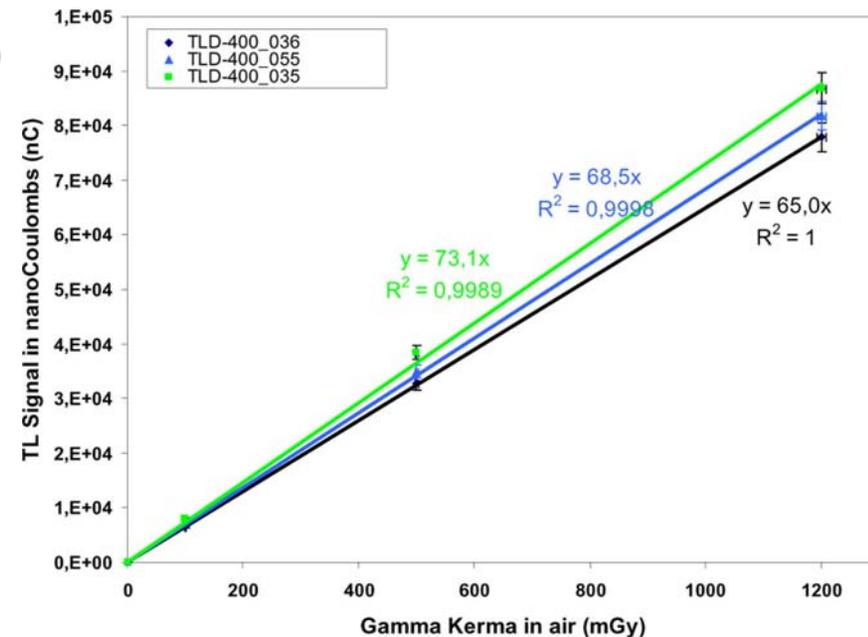
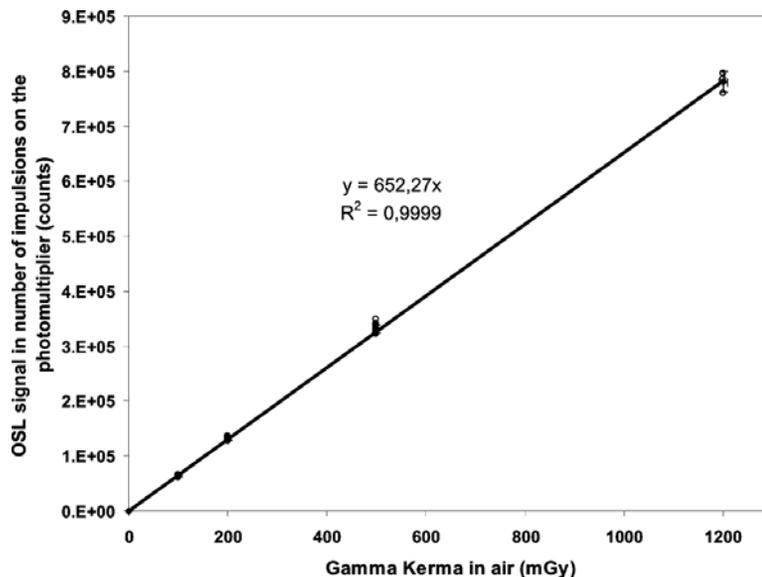
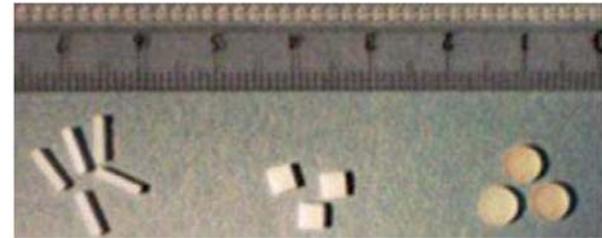
5 pillboxes holding 3  
detectors in the axial mid-  
plane of the core



H. Amharrak and al., Analysis and Recent Advances in Gamma Heating Measurements in MINERVE Facility by Using TLD and OSLD Techniques, IEEE Trans. Nucl. Sci. 59, 1360-1368, (2012)

## 1- Improvement of the calibration :

- Standard <sup>60</sup>Co source
- Al pillbox : electronic equilibrium
- Calibration curve
- Calibration uncertainty ~ 2% TLD/OSLD



## 2- Incore measurement : absorbed dose

- experimental uncertainty (calib., repeatability)
- neutron dose correction :

TLD : 8.6%

OSLD : 5.9%

## 3- 3D Monte Carlo simulation :

- cavity corection 2 steps calculation

TLD : 1.07

OSLD : 1.05

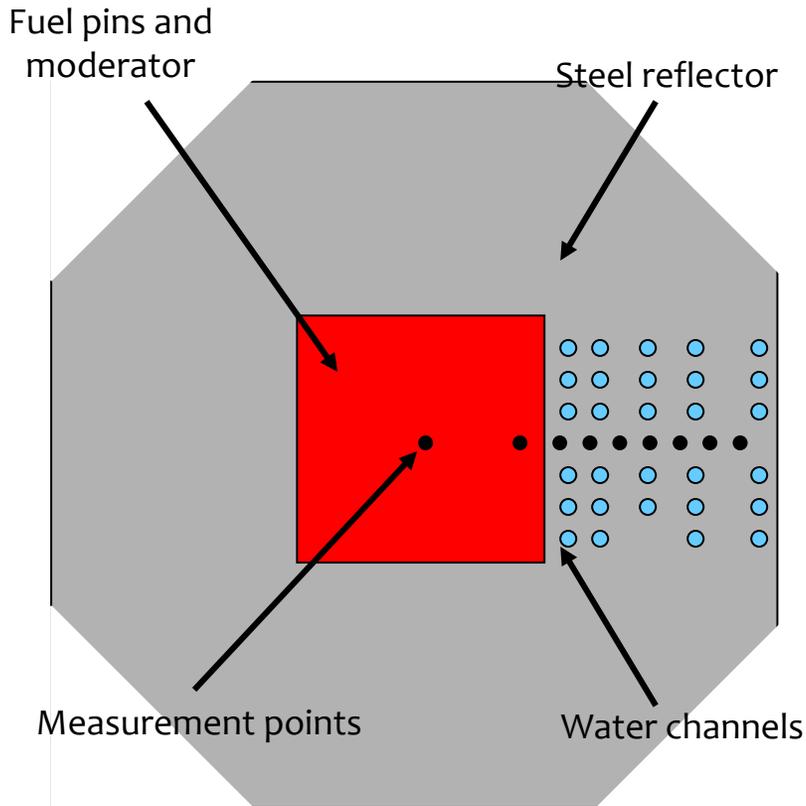
- Delayed  $\gamma$  contribution : 32%

C/E (k=2)

TLD :  $1.05 \pm 5.3\%$

OSLD :  $0.96 \pm 7.0\%$

discrepancy TLD/OSLD ?  
n dose, delayed  $\gamma$   
simulation methods



## PERLE in EOLE facility

- A 22 cm stainless steel neutron reflector
- TLD gamma heating measurement in & out-core
- 27 X 27 3.7%  $^{235}\text{U}$  enriched fuel pins



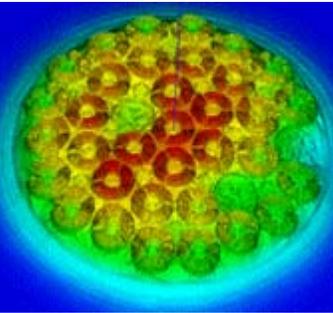
Fe,  $^{62}\text{Ni}$  new evaluations qualification

# INTEGRAL EXPERIMENTS AND INSTRUMENTATION : JHR general characteristics

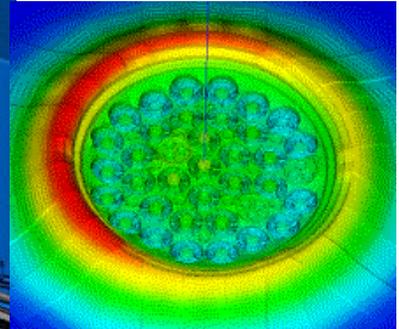
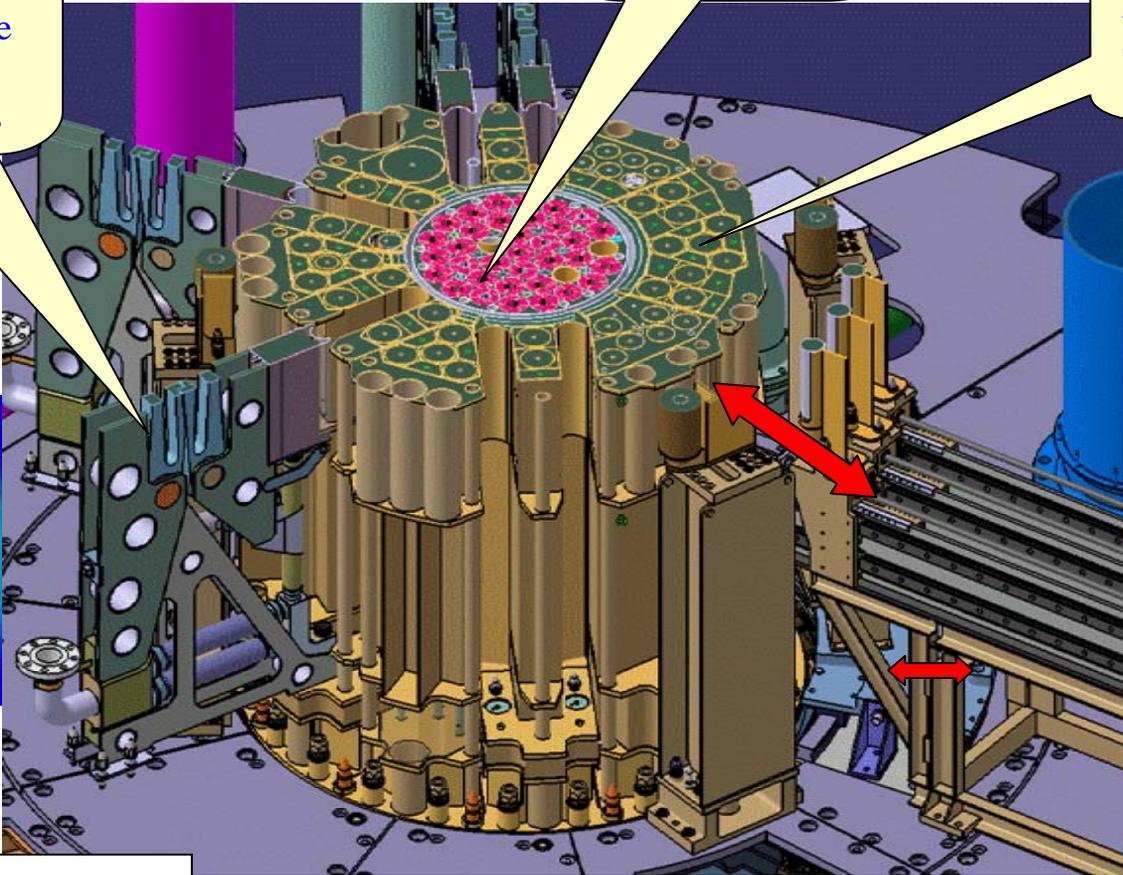
Displacement systems:  
 • To adjust the fissile power  
 • To study transients

Material ageing  
 (up to 16 dpa/y)

Fuel studies: up to  
 600 W/cm with a  
 1% <sup>235</sup>U PWR rod



**Fast neutron flux**

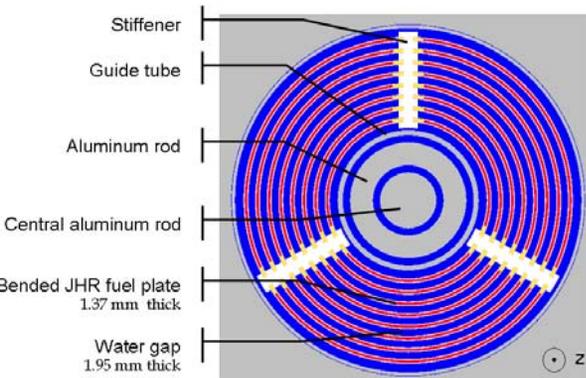
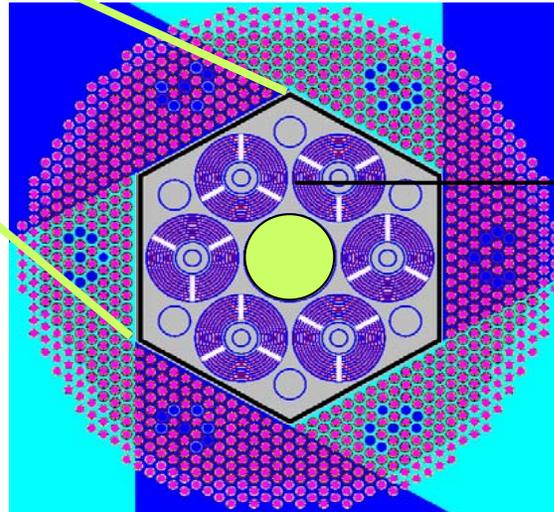
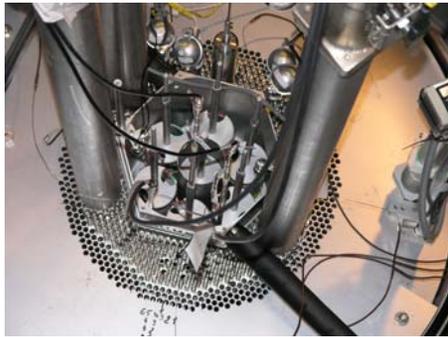
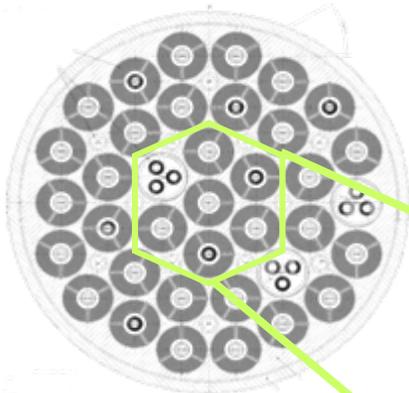


**Thermal neutron flux**

**In reflector**  
 Up to  $5.5 \cdot 10^{14} \text{ n.cm}^{-2}.\text{s}^{-1}$   
 Fixed positions and  
 displacement systems

**In core**  
 Up to  $5.5 \cdot 10^{14} \text{ n/cm}^2.\text{s} > 1\text{MeV}$   
 Up to  $10^{15} \text{ n/cm}^2.\text{s} > 0.1\text{MeV}$

# INTEGRAL EXPERIMENTS AND INSTRUMENTATION : AMMON in EOLE



- Experiment characteristics:
  - Central experimental zone : aluminum rack
  - $U_3Si_2$ -Al fuel ( $e=27\%$ )
  - Driver zone : 623 UOx fuel pins ( $e=3.7\%$ )

## • Main Configurations:

- Reference core : 7 JHR assemblies



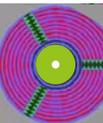
- Hf rod: completely inserted, 1/2 inserted



- Beryllium



- Voided assembly



**Experimental data:** critical state, reactivity worth, power distributions, spectrum indices, conversion factor,  $T^\circ$  coefficient, **gamma-heating**

## AMMON EXPERIMENTAL PROGRAM IN EOLE FACILITY

### Experiment in progress

### gamma-heating measurements :

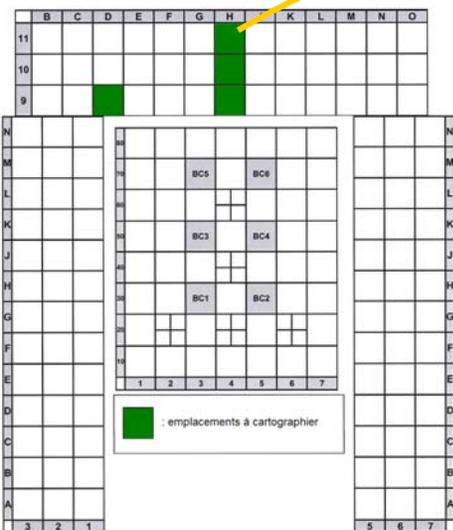
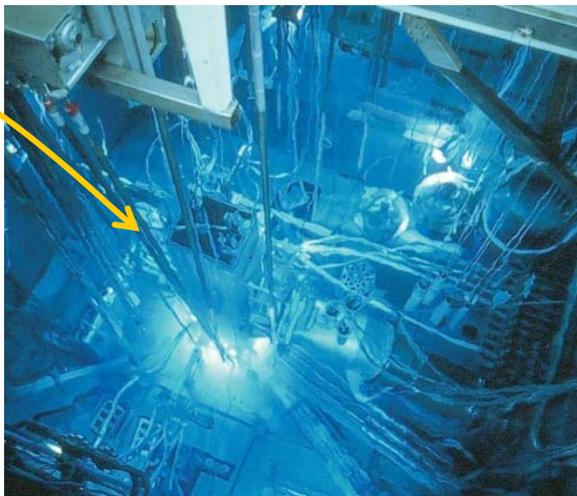
- . TLDs (7LiF, CaF<sub>2</sub>:Mn) , OSLDs (Al<sub>2</sub>O<sub>3</sub>:C ) , ionization chamber
- . Where ?
  - assembly center,
  - aluminum filler,
  - hafnium rod,
  - beryllium block

### Validation :

deposited energy in **aluminum**, **hafnium** and **beryllium**  
for JHR safety and performance study

### Interpretation : (TRIPOLI4/JEFF3):

- . Collaboration work in progress
- . PhD work starting at the end of 2012



## CARMEN-1 in OSIRIS facility

- Neutron & photon measurements
- 2 experimental devices dedicated
- 4 out-core measuring locations scanned in 70 MW OSIRIS reflector (CEA-Saclay)

**$n,\gamma$  combined analysis => JHR core mapping**

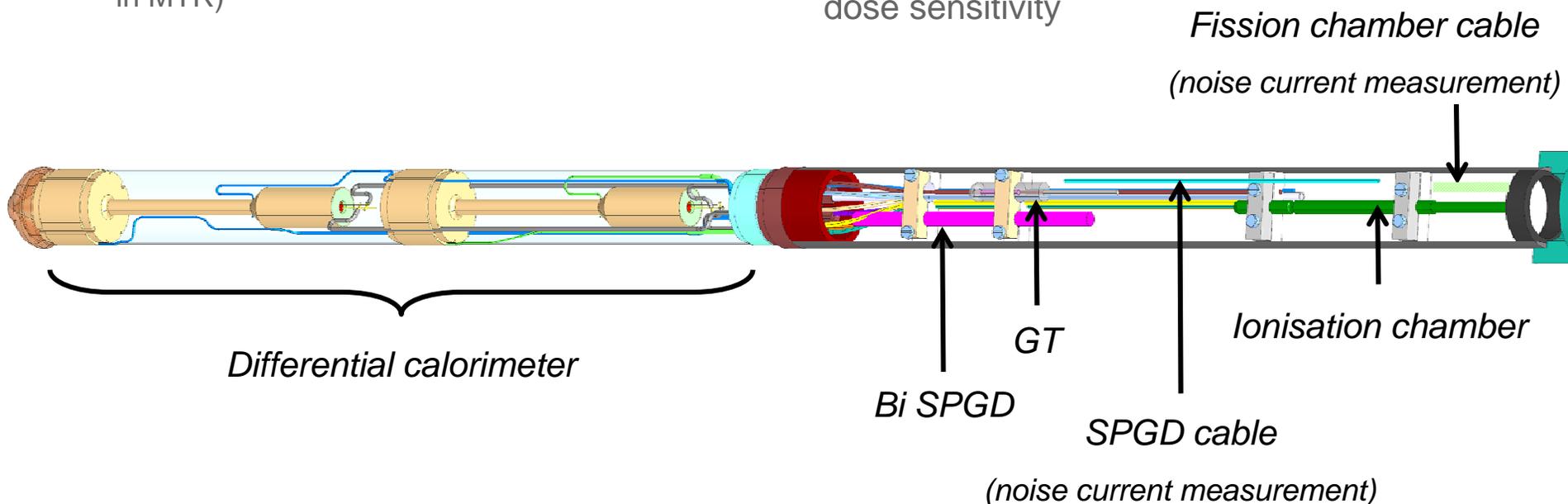
*D. Fourmentel and al., Combined analysis of neutron and photon flux measurements for Jules Horowitz reactor core mapping, Conf. PHYTRA2, (2011)*

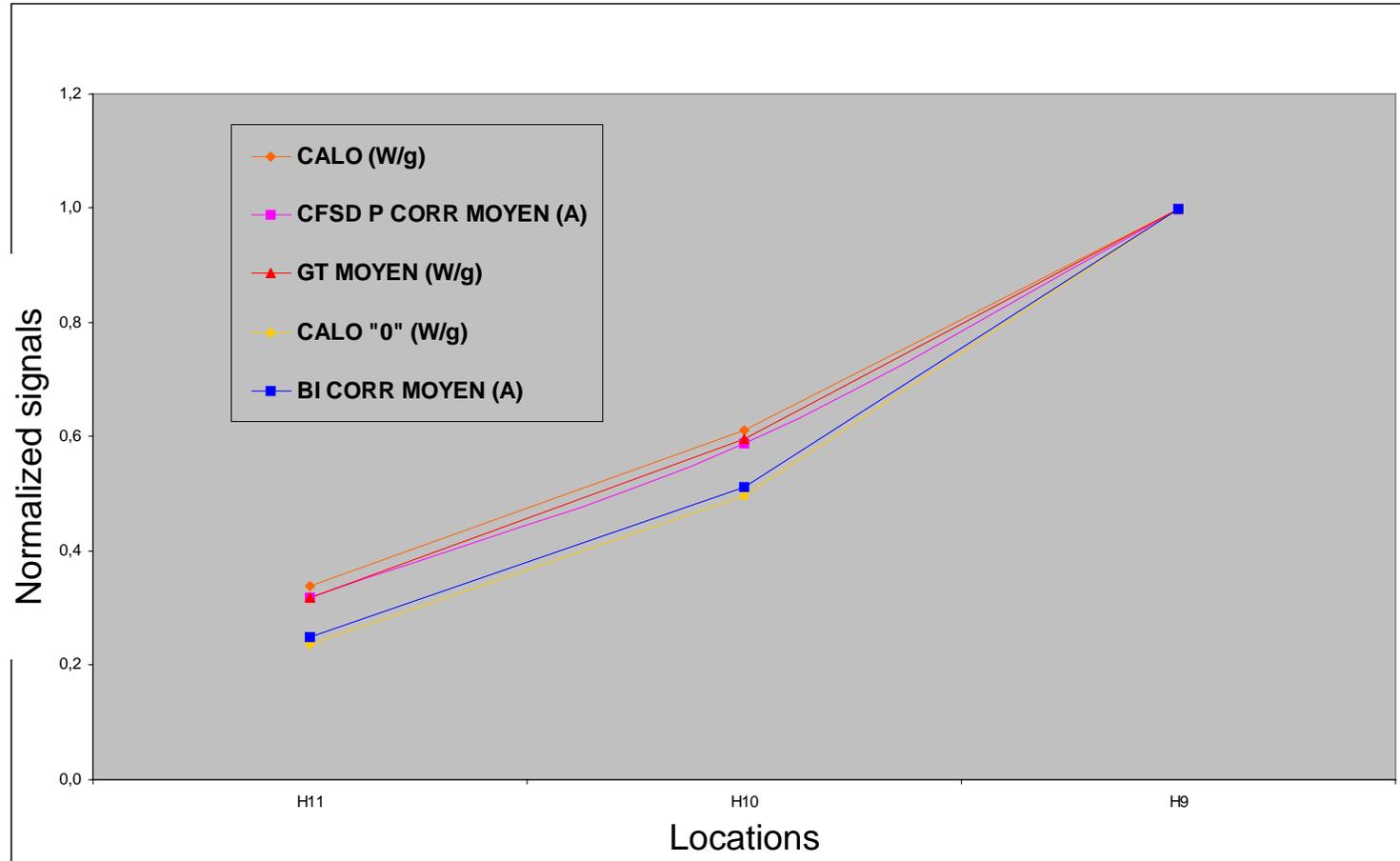
## CARMEN1-P

- Gamma flux :  
Ionization chamber,  
Self Powered Gamma Detector (SPGD)
- Nuclear heating :  
Differential calorimeter,  
Gamma Thermometer (commonly used  
in MTR)

## OBJECTIVES

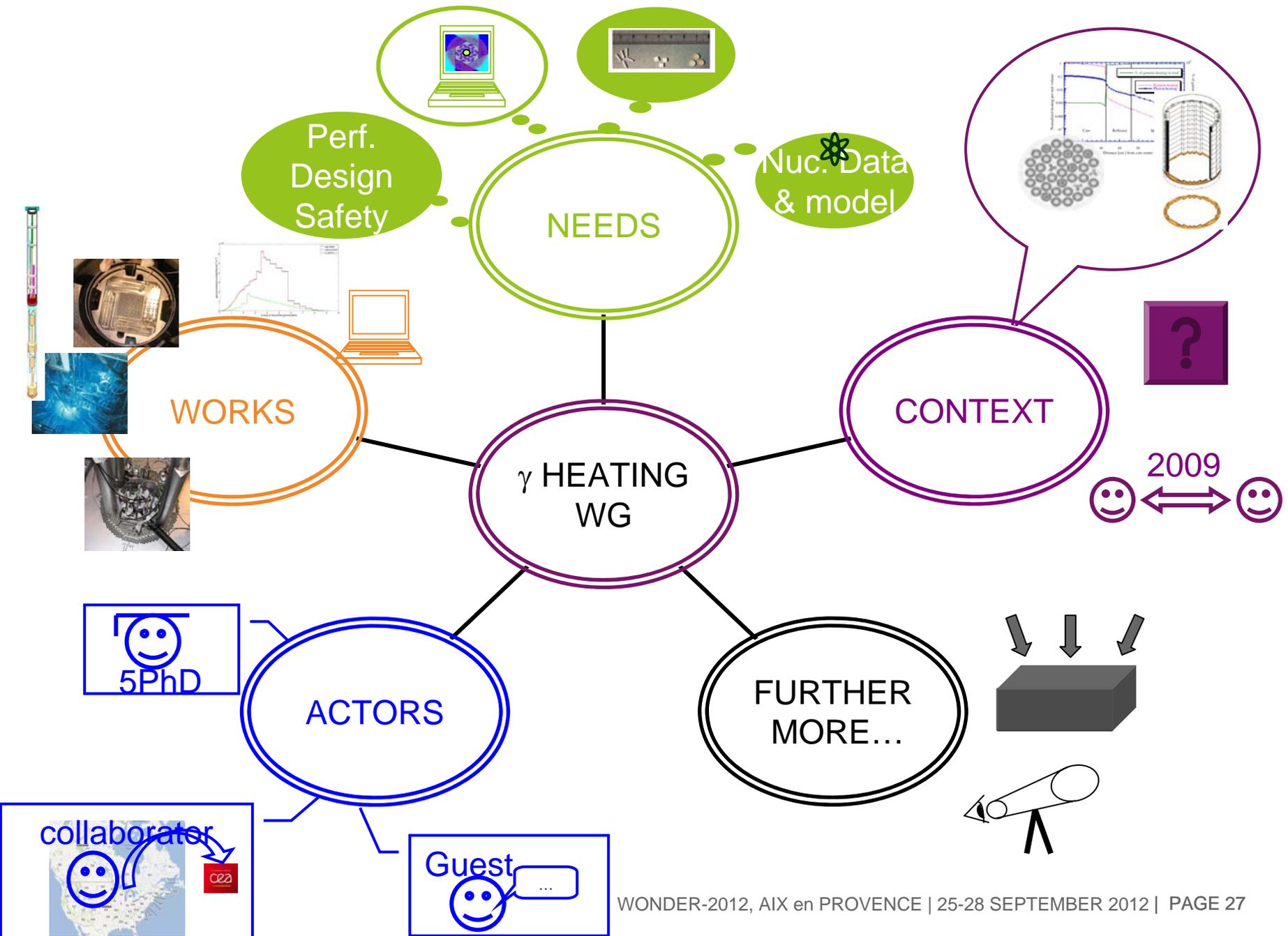
- Reduce measurement uncertainties
- Testing 3 different methods to  
estimate nuclear heating from  
temperature measurement
- 2 sensors with different material  
(graphite vs stainless steel) & neutron  
dose sensitivity





Signals evolution of gamma sensors between different locations

## **5. CONCLUSION**



# Thank You

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Centre de Saclay | 91191 Gif-sur-Yvette Cedex  
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